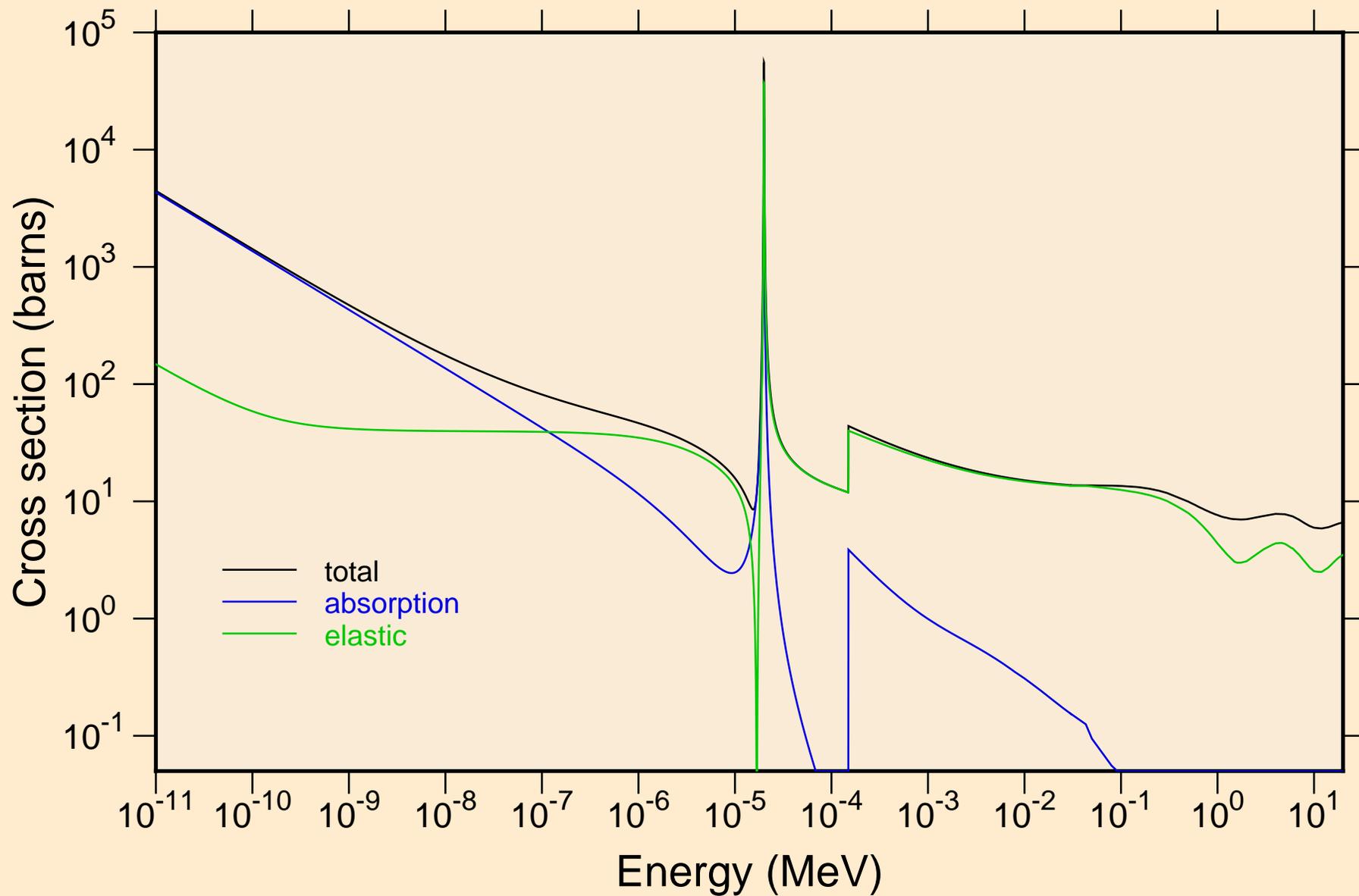
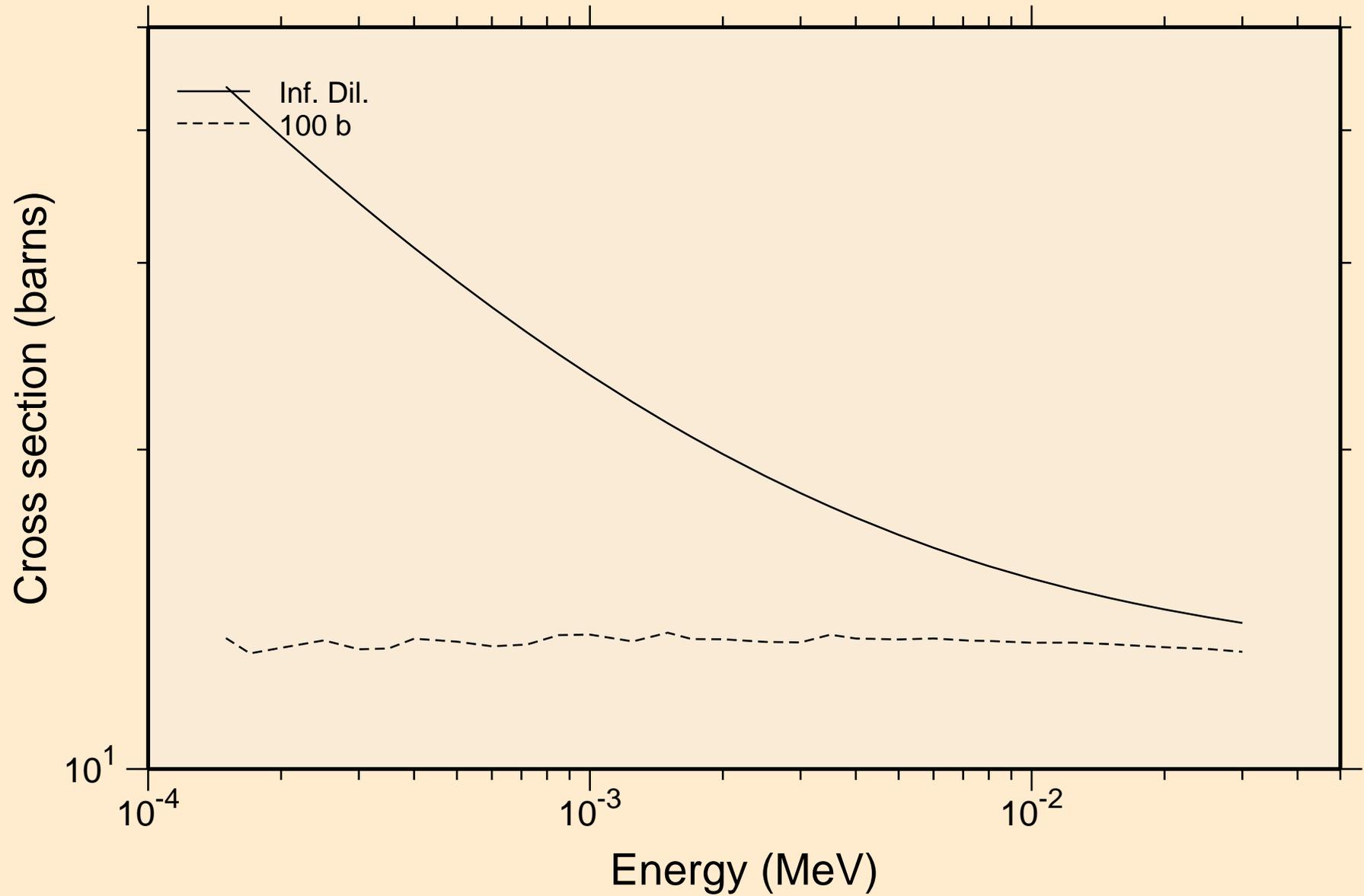


# ENDF/B-VII CM-250

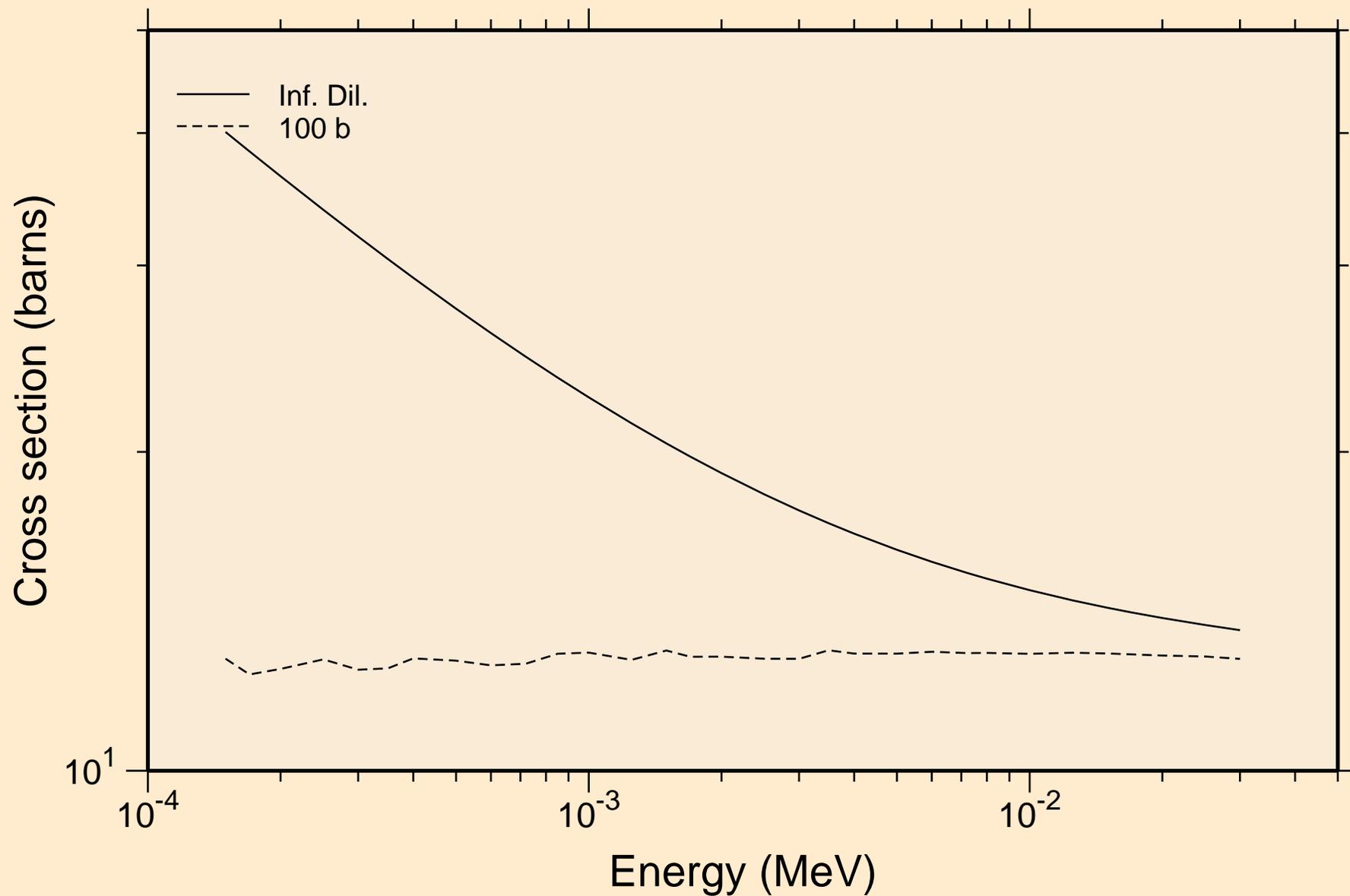
## Principal cross sections



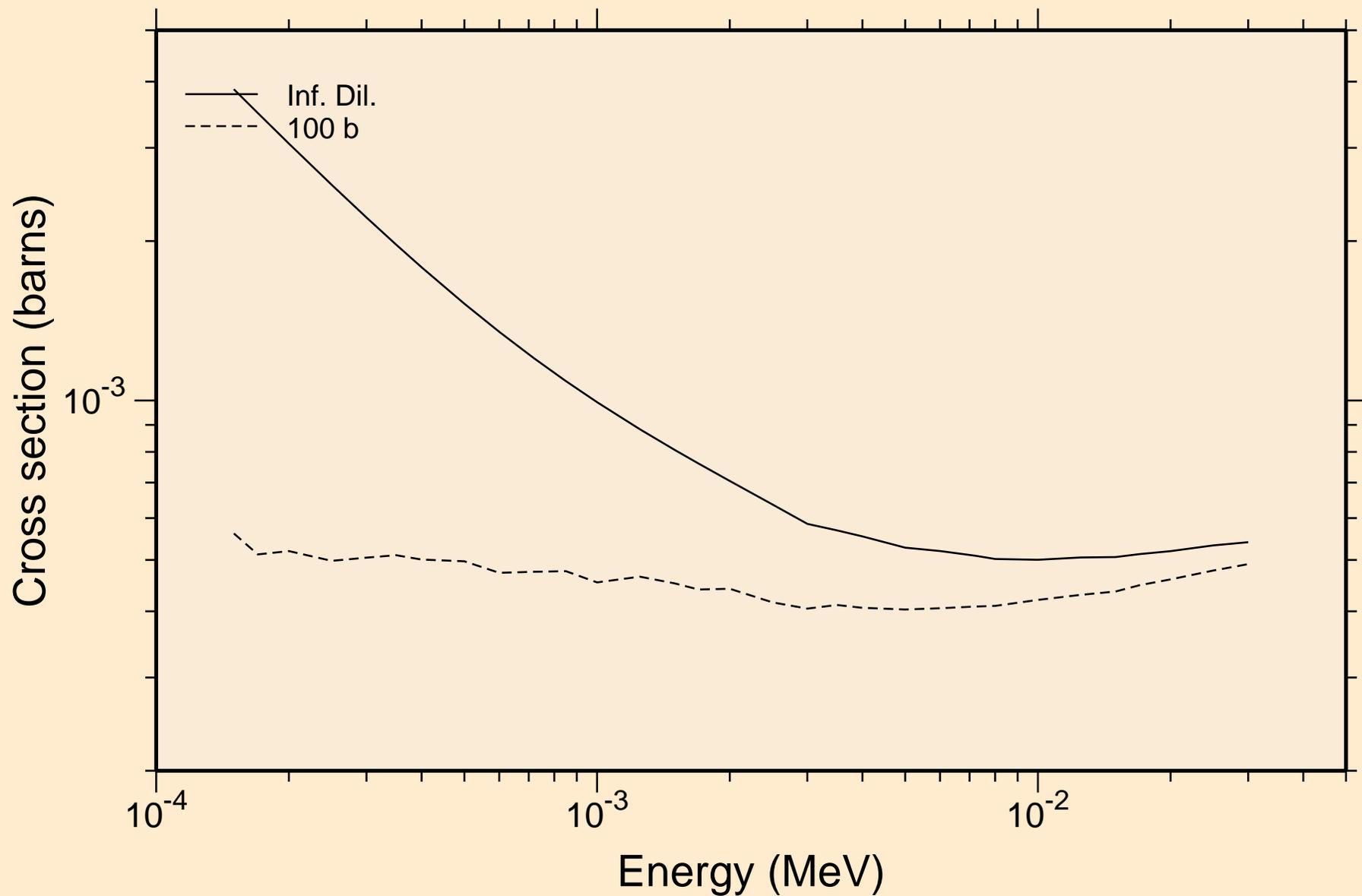
ENDF/B-VII CM-250  
UR total cross section



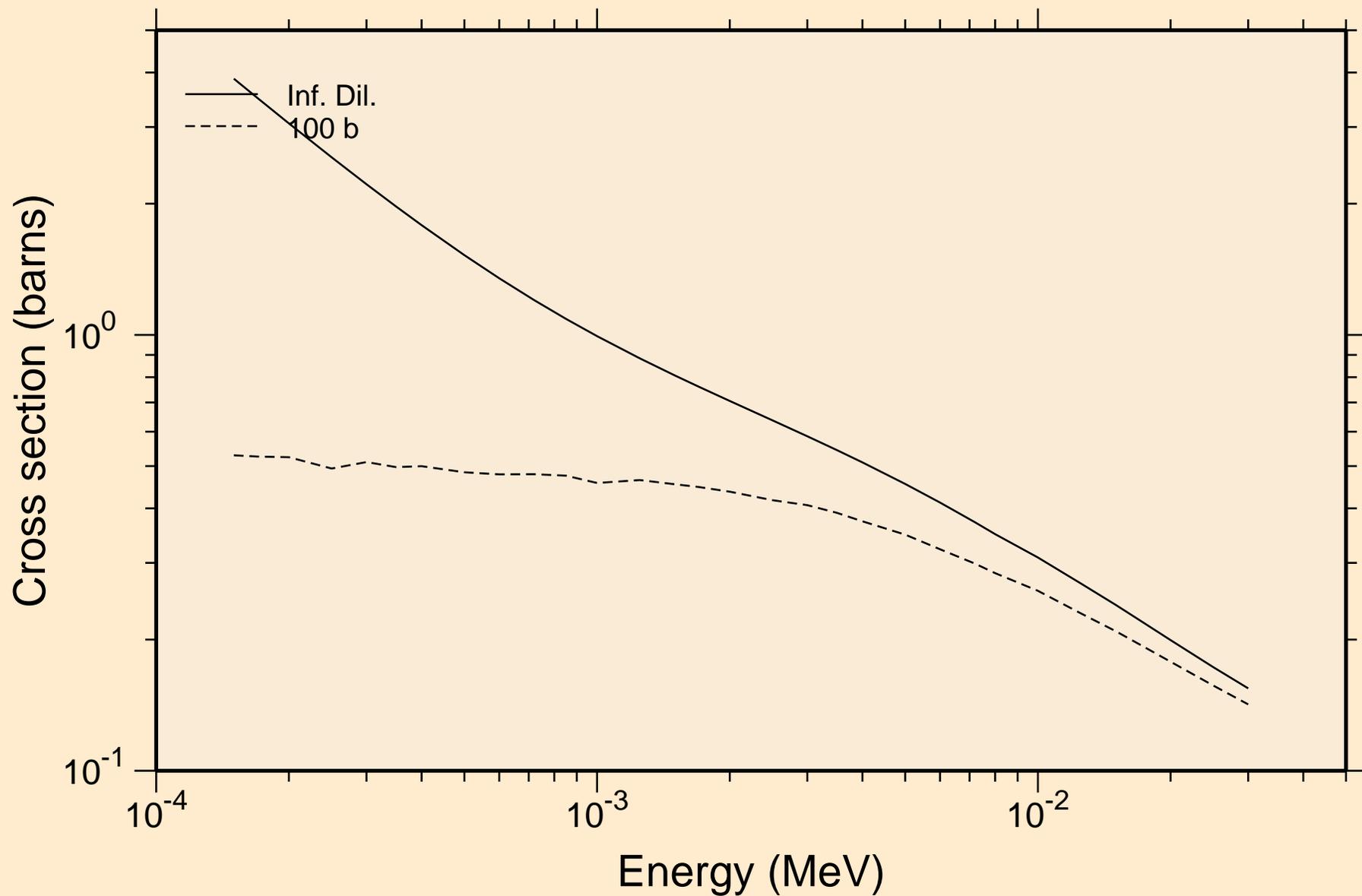
ENDF/B-VII CM-250  
UR elastic cross section



ENDF/B-VII CM-250  
UR fission cross section

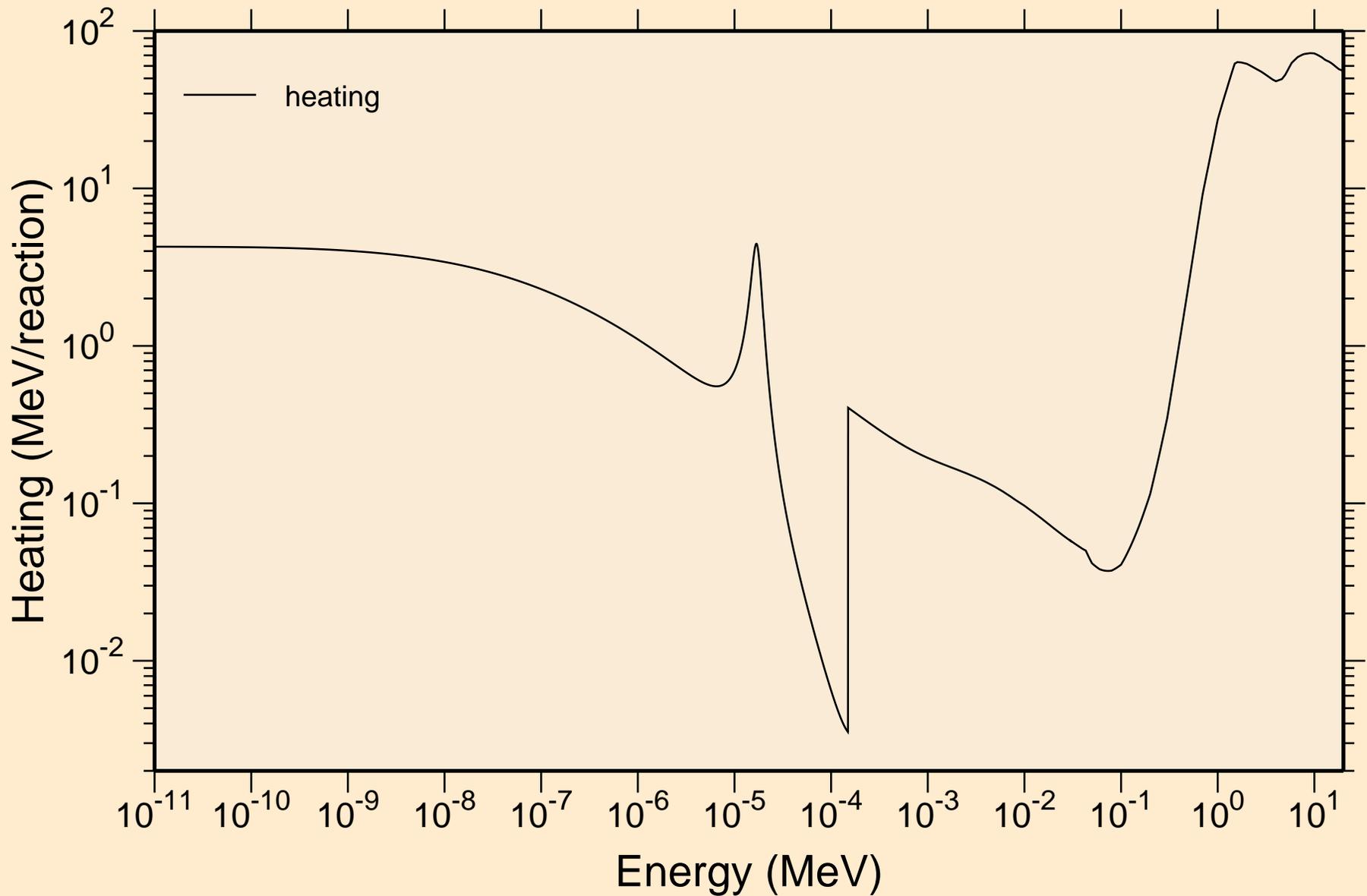


ENDF/B-VII CM-250  
UR capture cross section



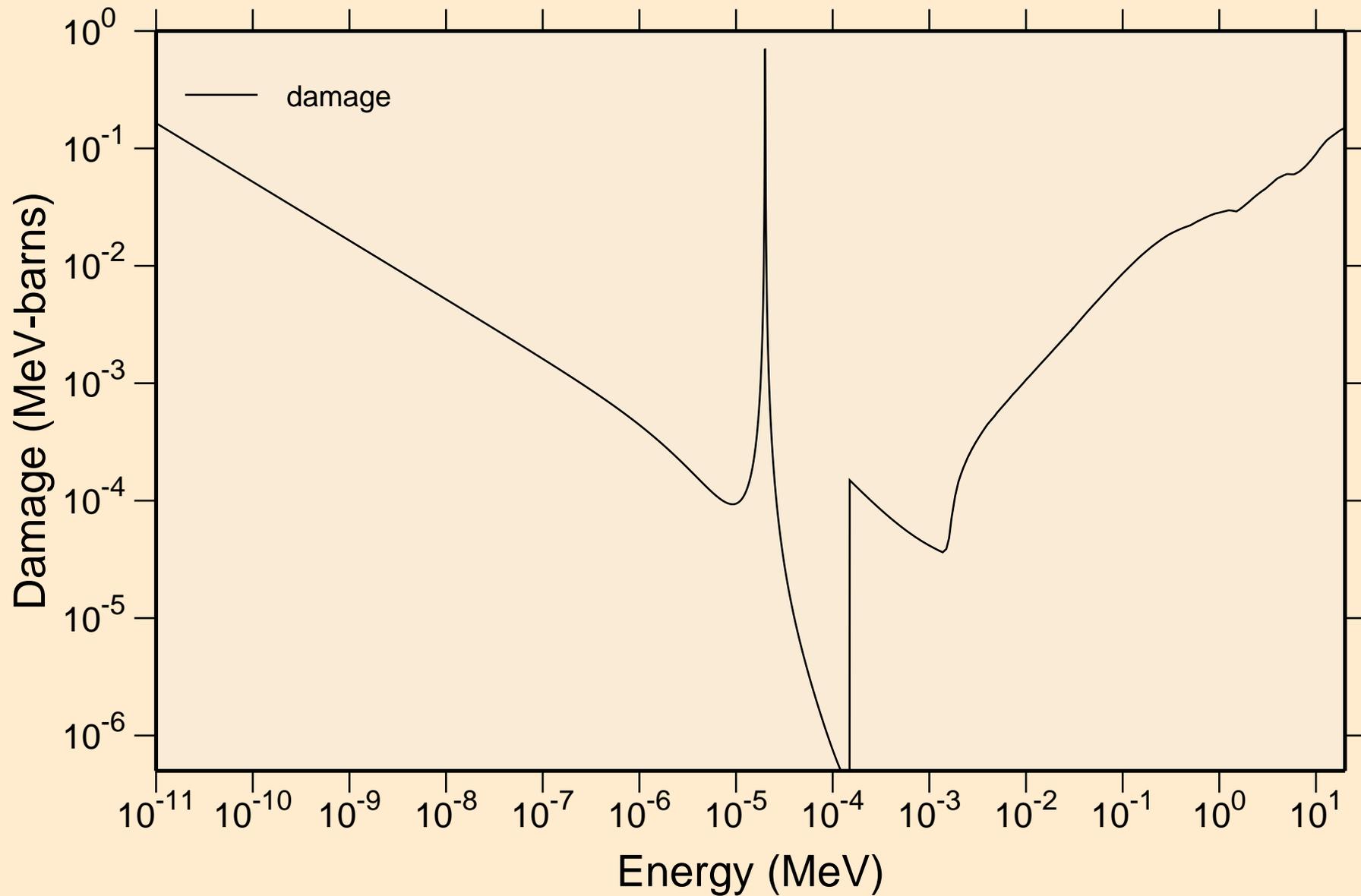
# ENDF/B-VII CM-250

## Heating



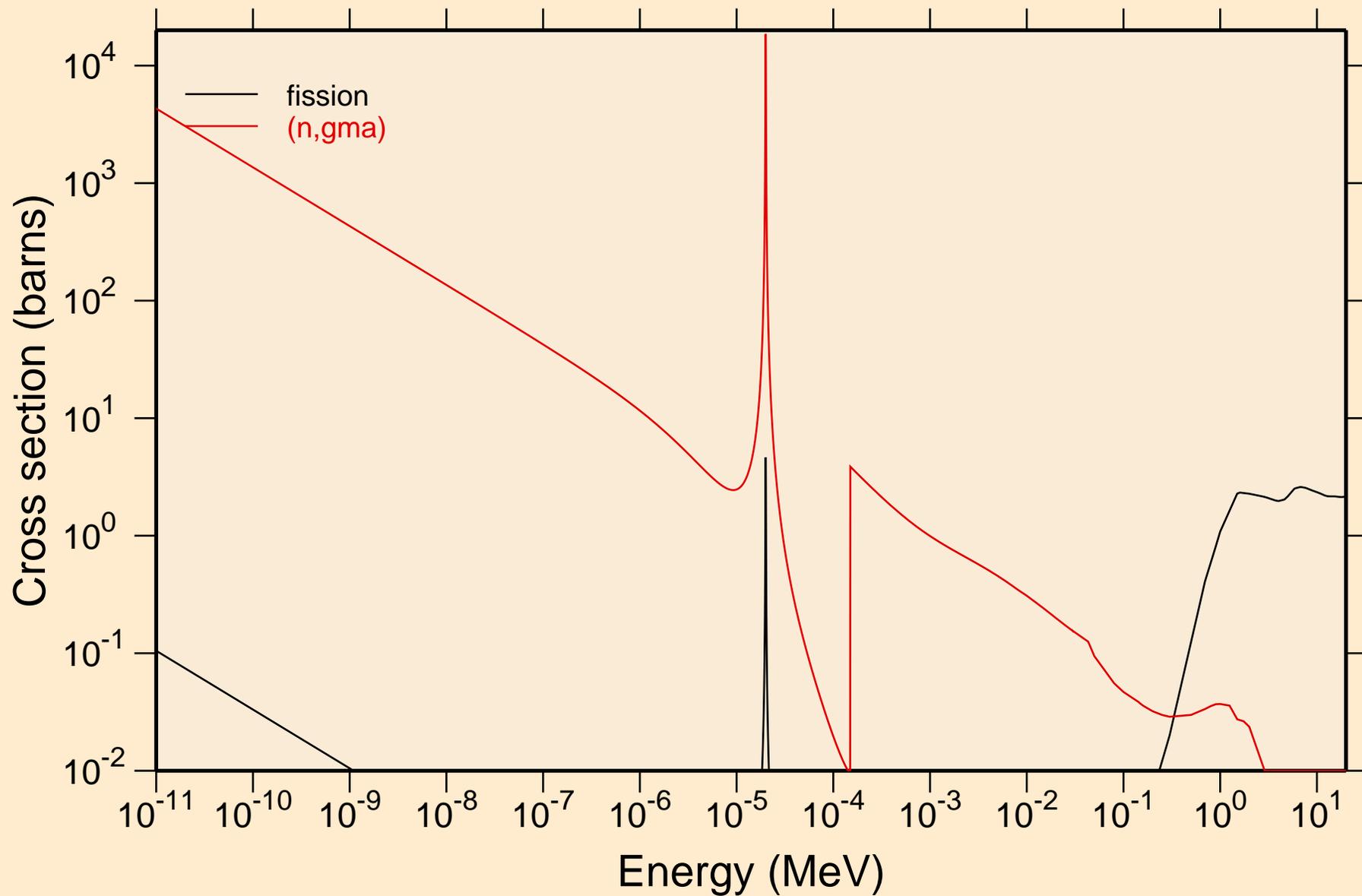
# ENDF/B-VII CM-250

## Damage



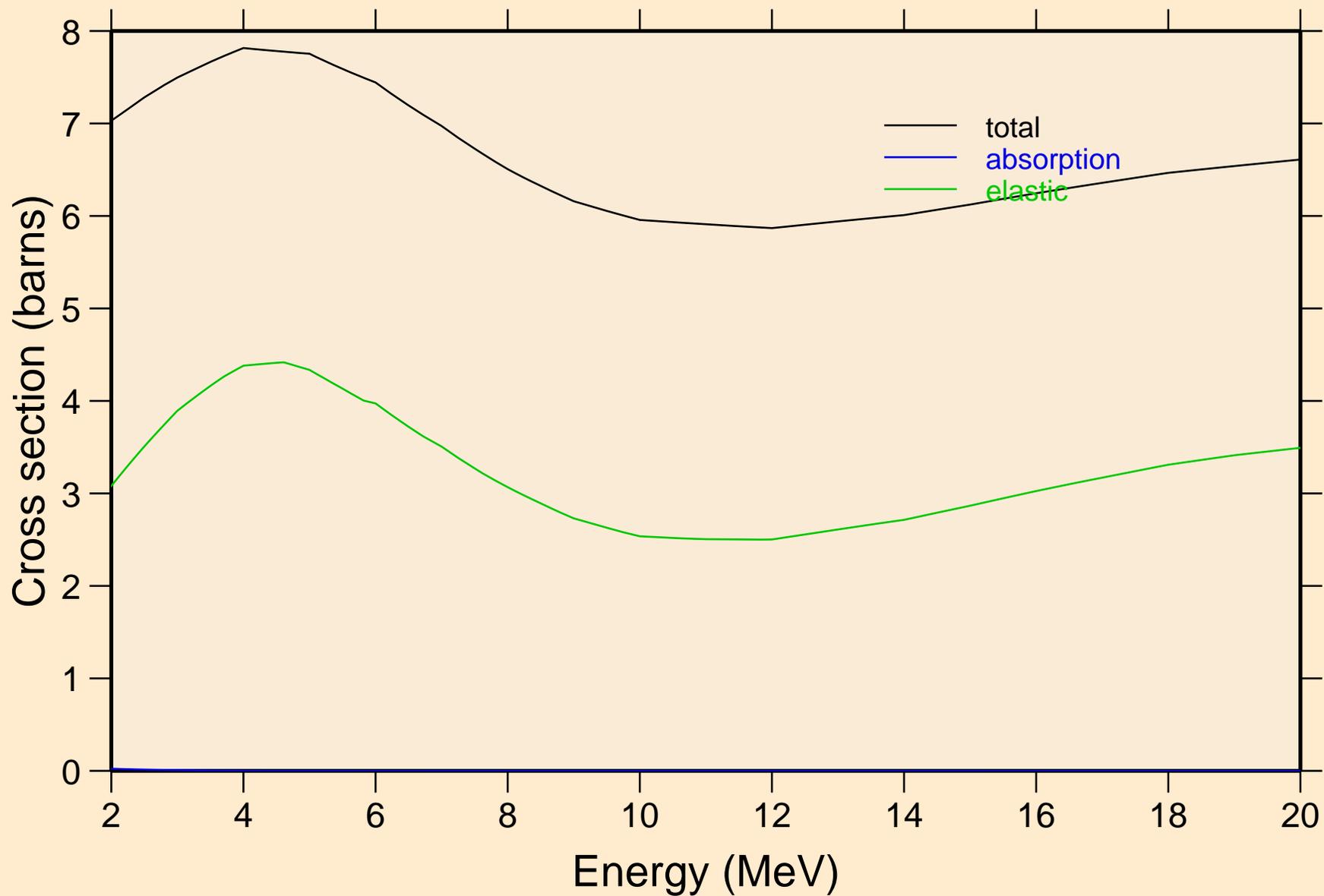
# ENDF/B-VII CM-250

## Non-threshold reactions



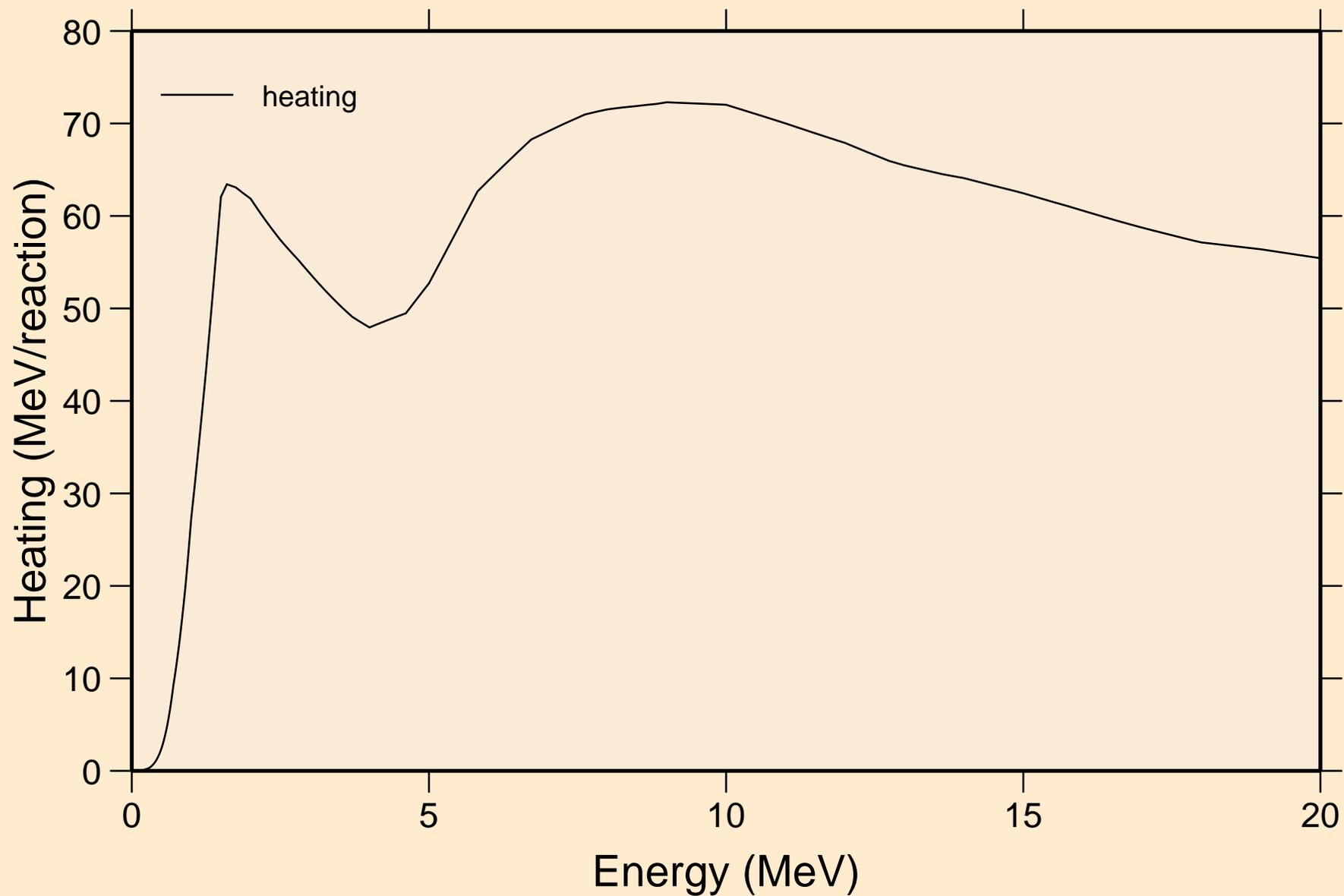
# ENDF/B-VII CM-250

## Principal cross sections



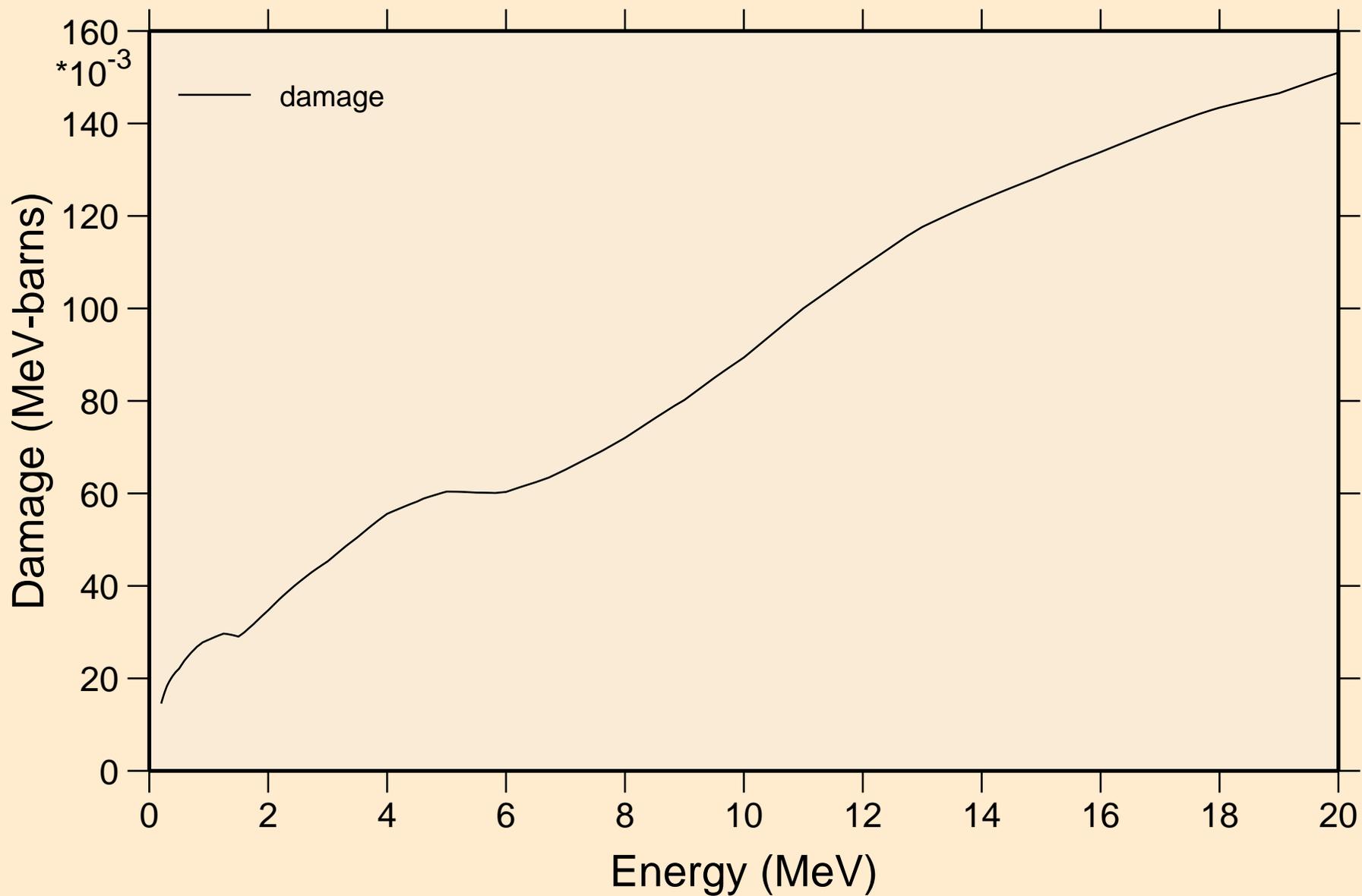
# ENDF/B-VII CM-250

## Heating



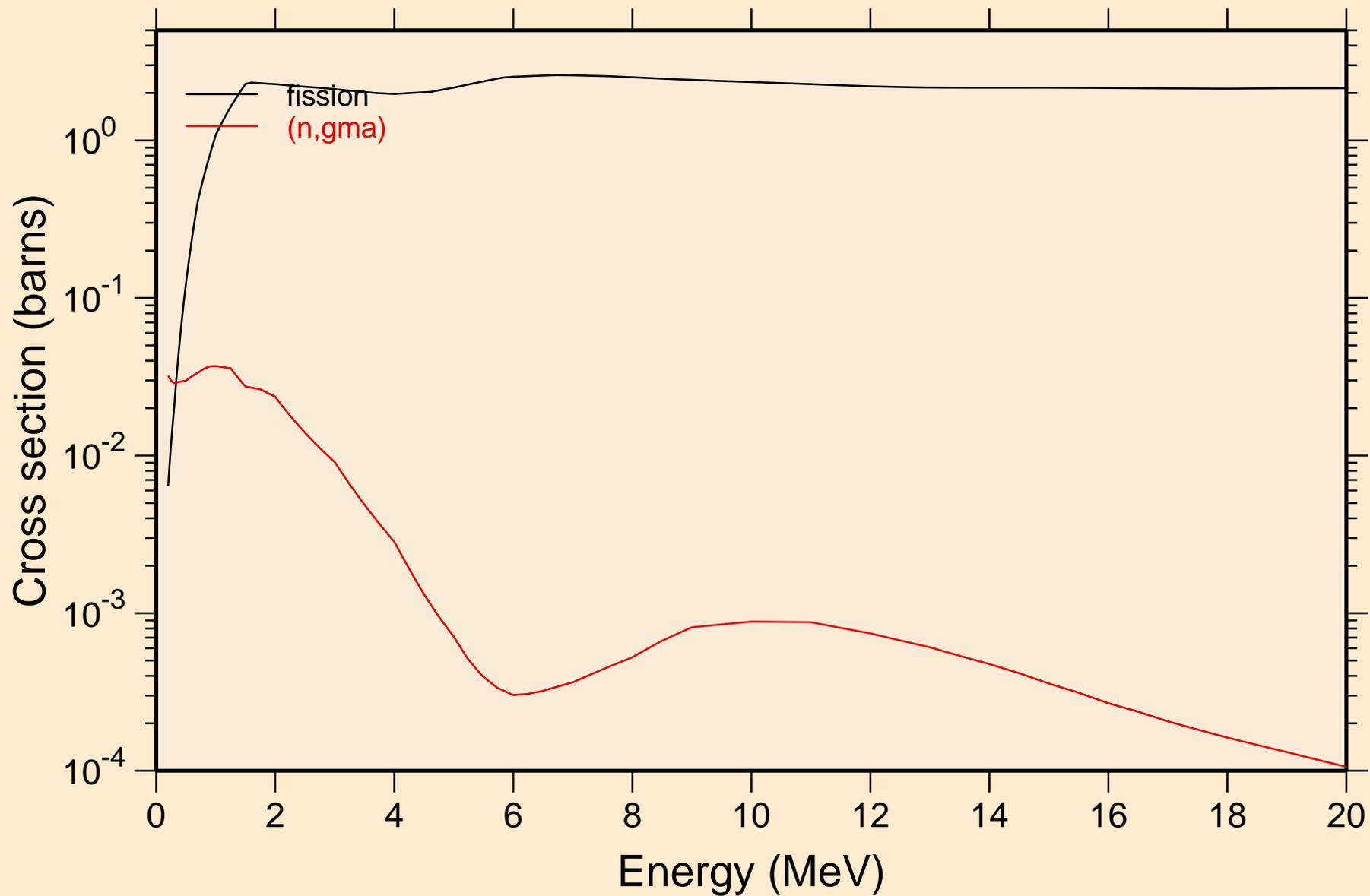
# ENDF/B-VII CM-250

## Damage



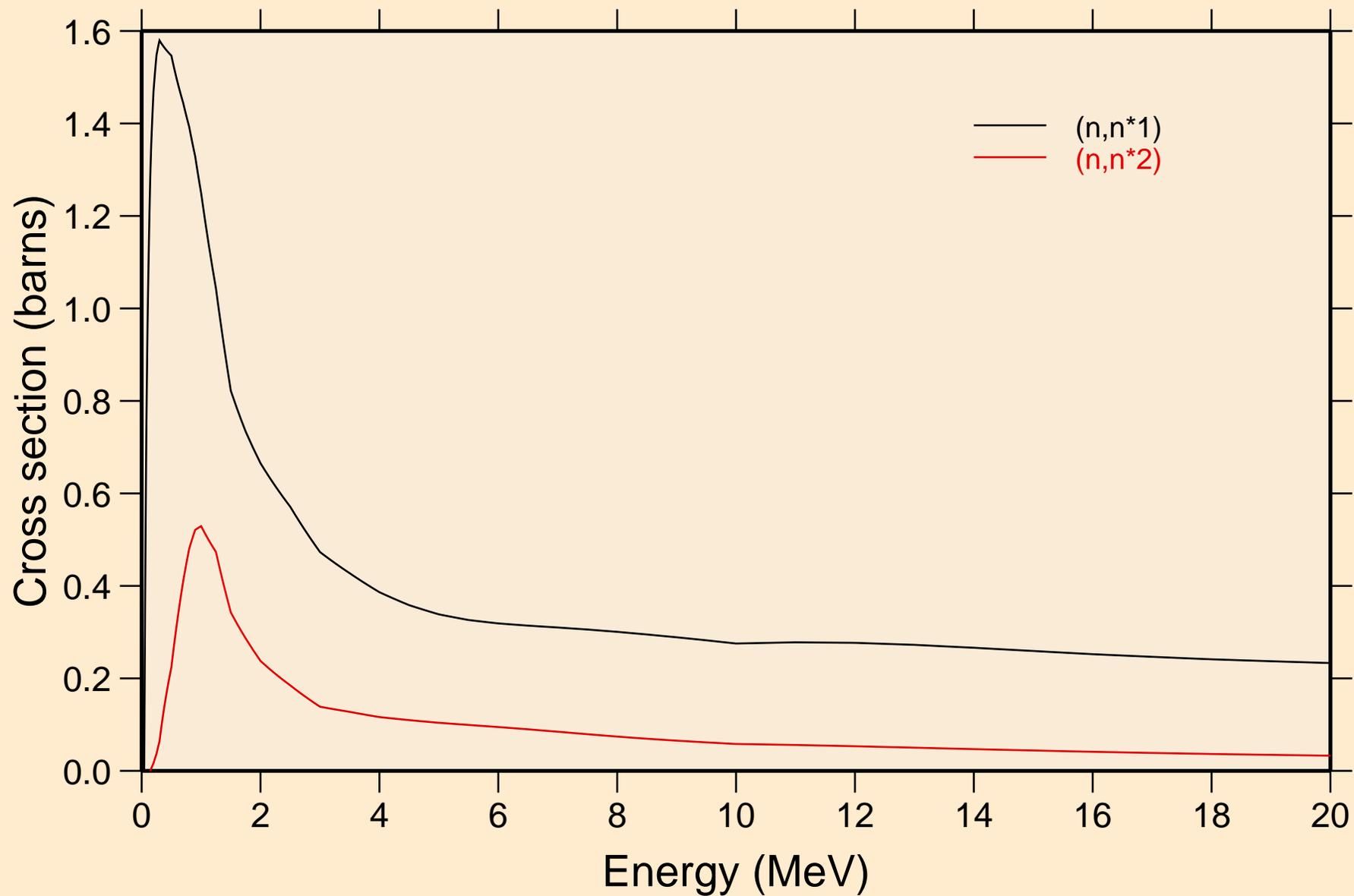
# ENDF/B-VII CM-250

## Non-threshold reactions



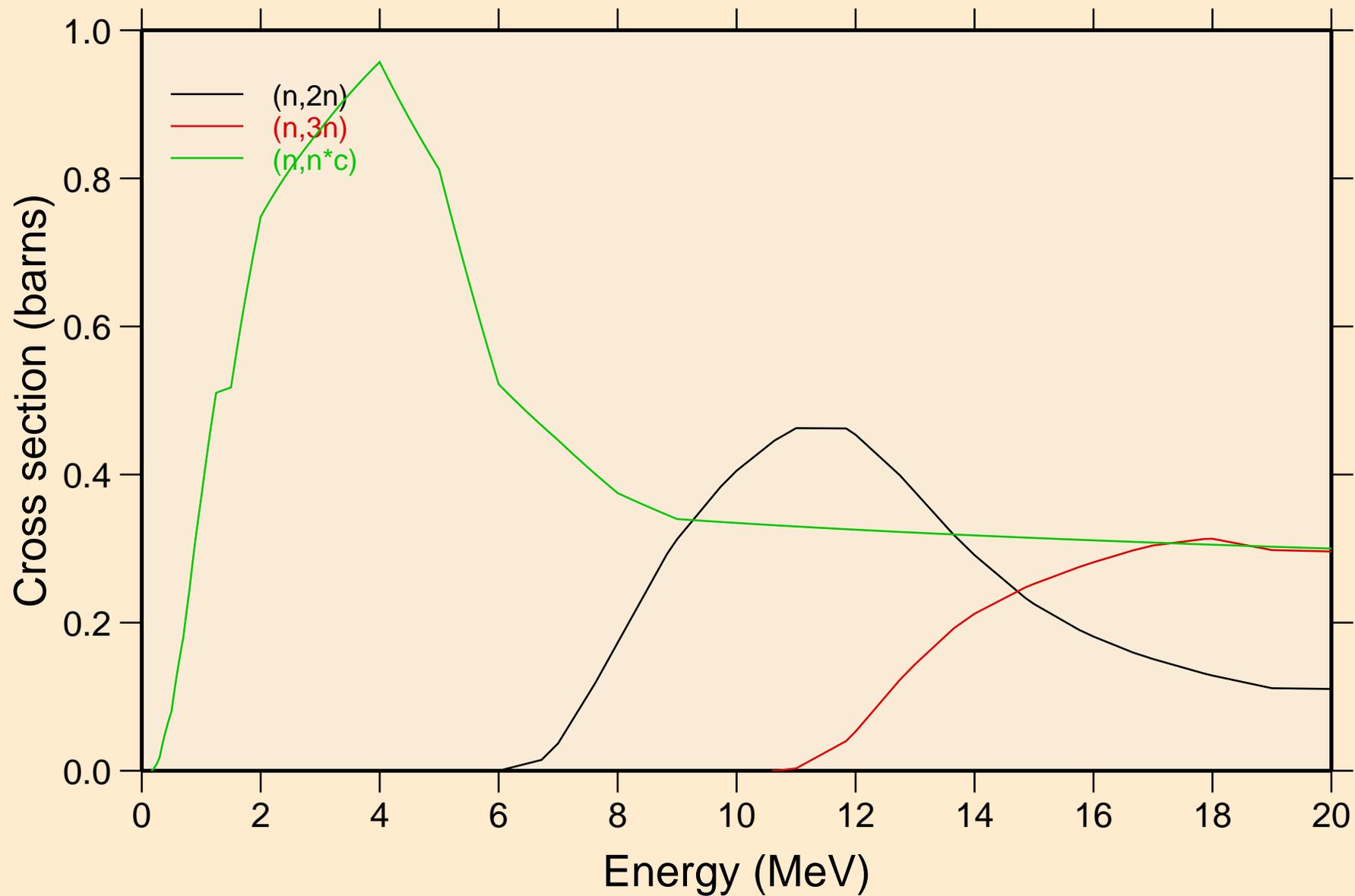
# ENDF/B-VII CM-250

## Inelastic levels

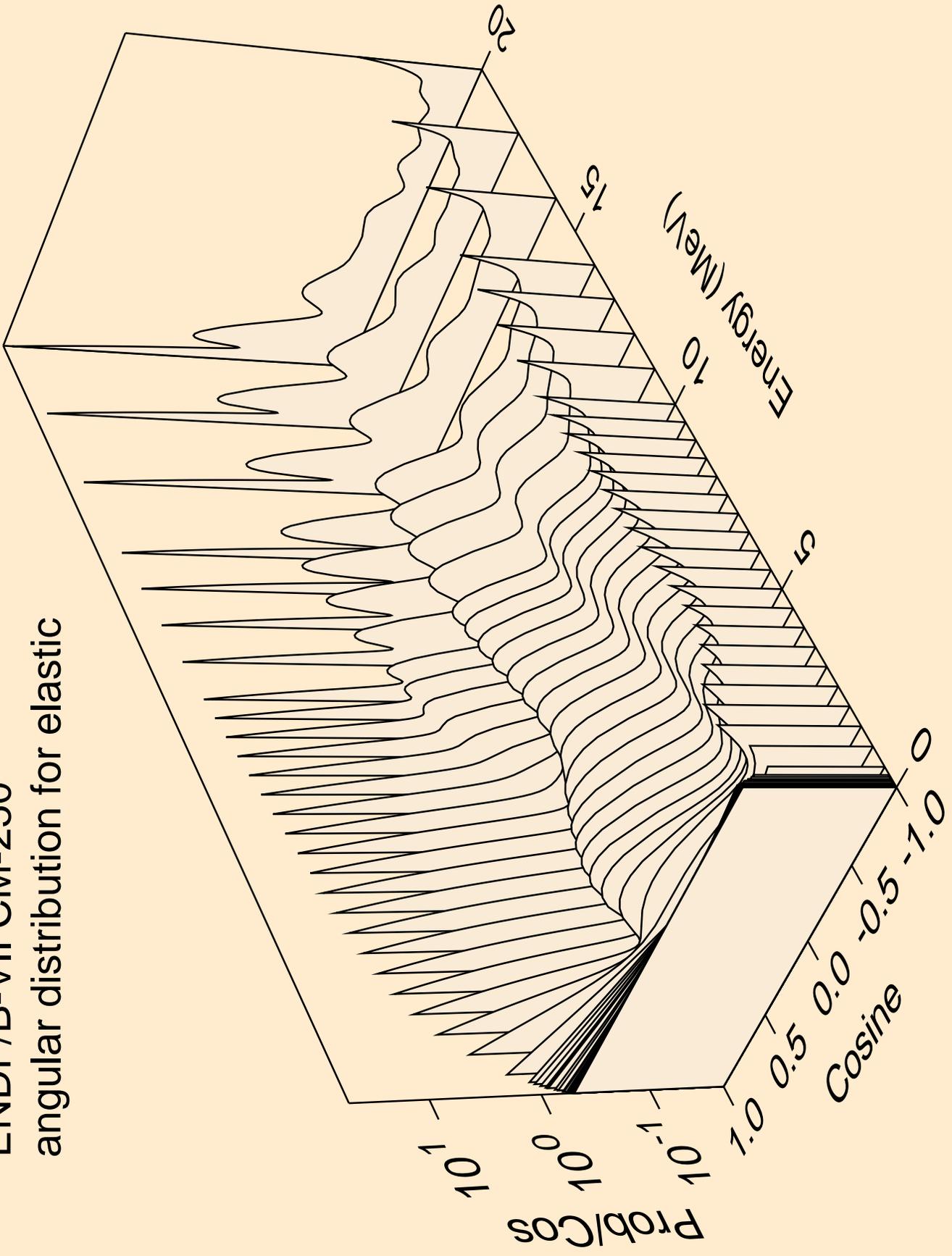


# ENDF/B-VII CM-250

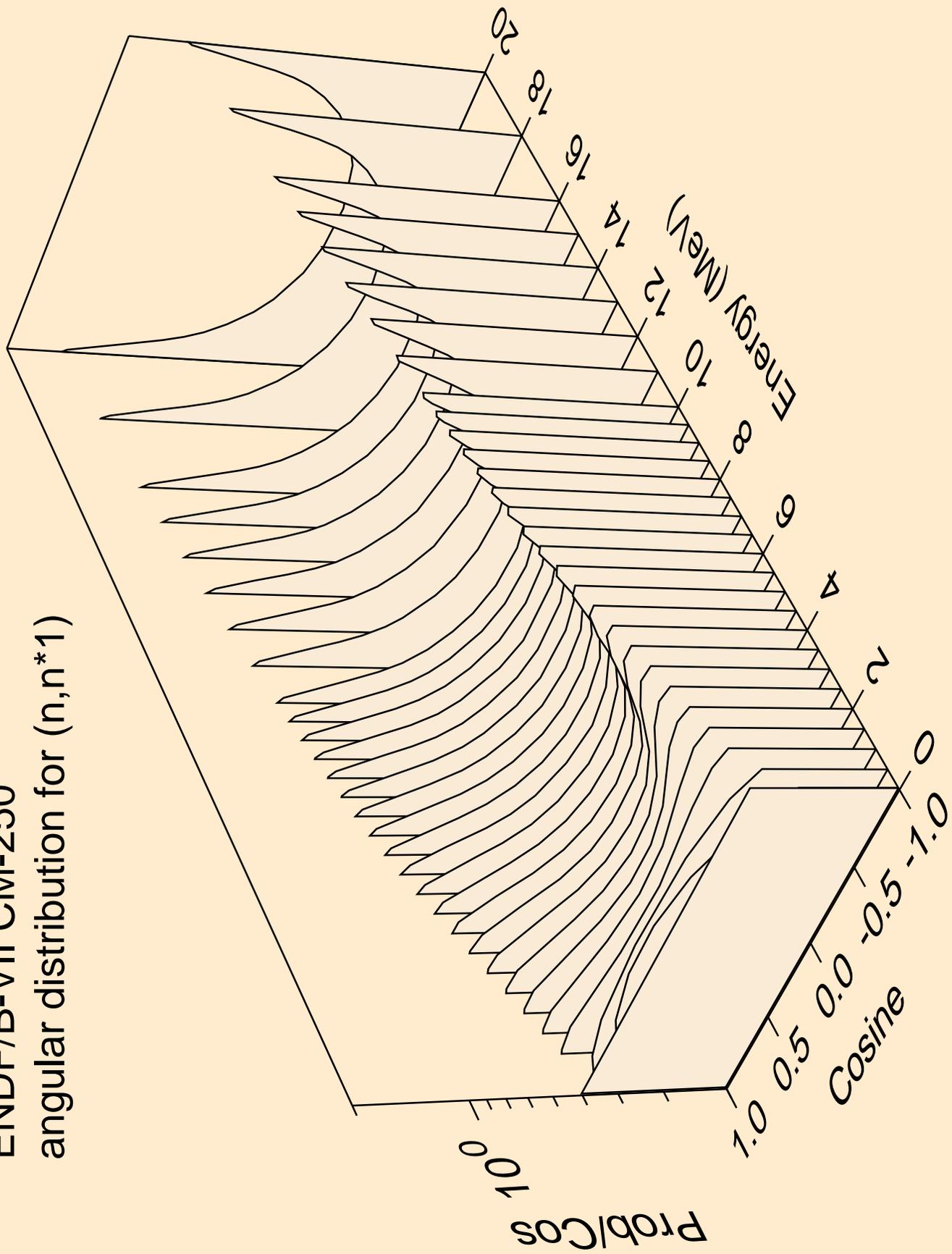
## Threshold reactions



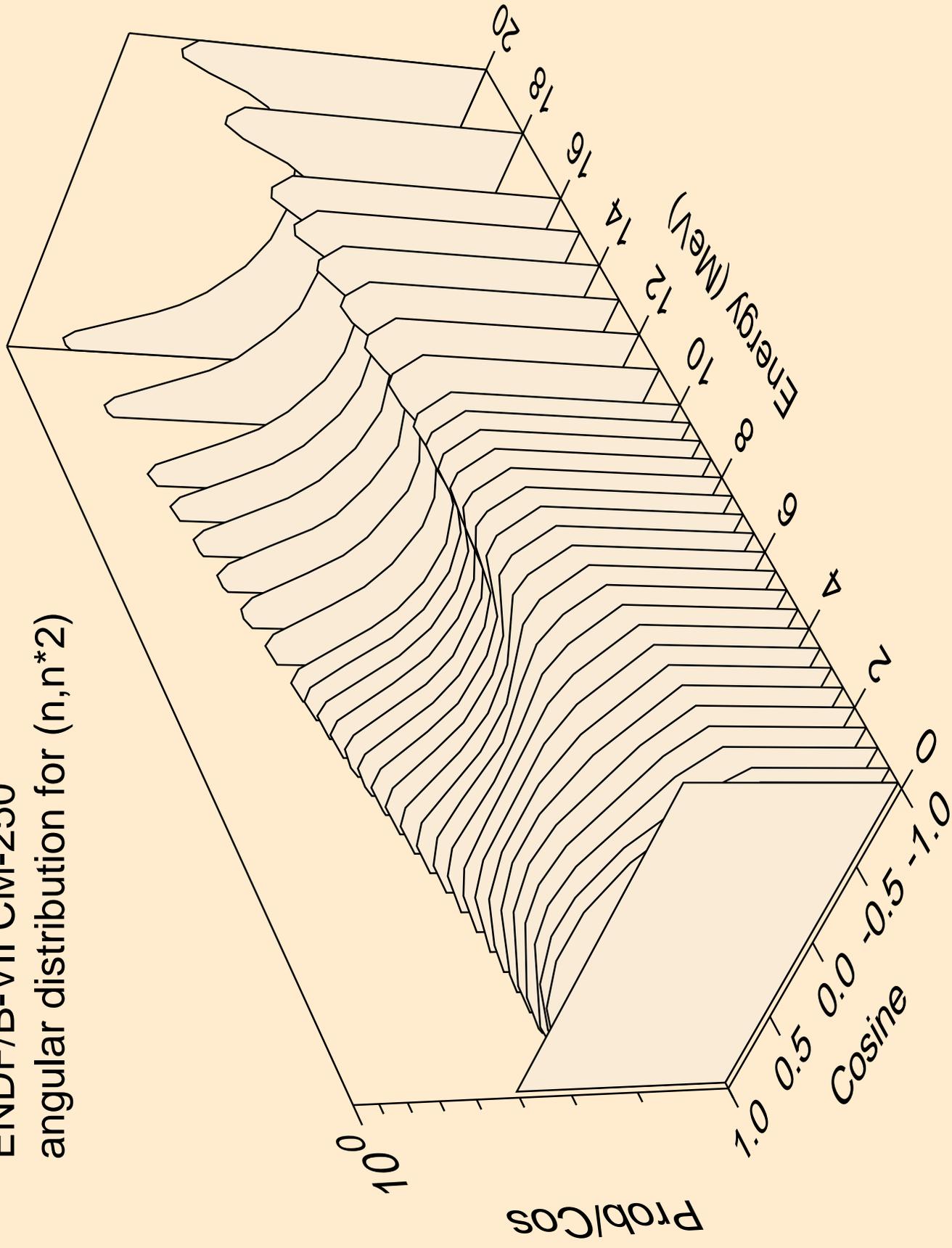
ENDF/B-VII CM-250  
angular distribution for elastic



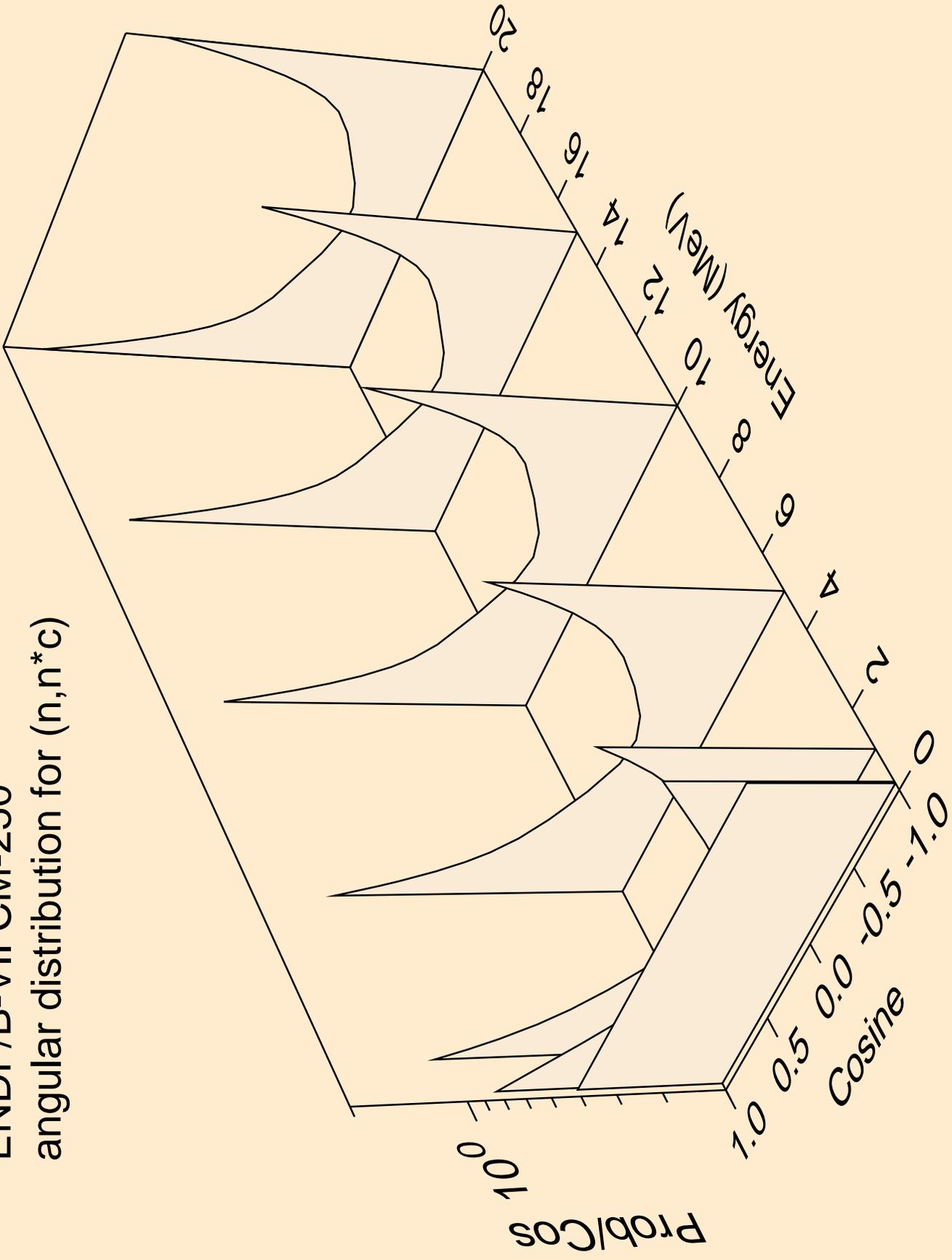
ENDF/B-VII CM-250  
angular distribution for (n,n\*1)



ENDF/B-VII CM-250  
angular distribution for (n,n\*2)

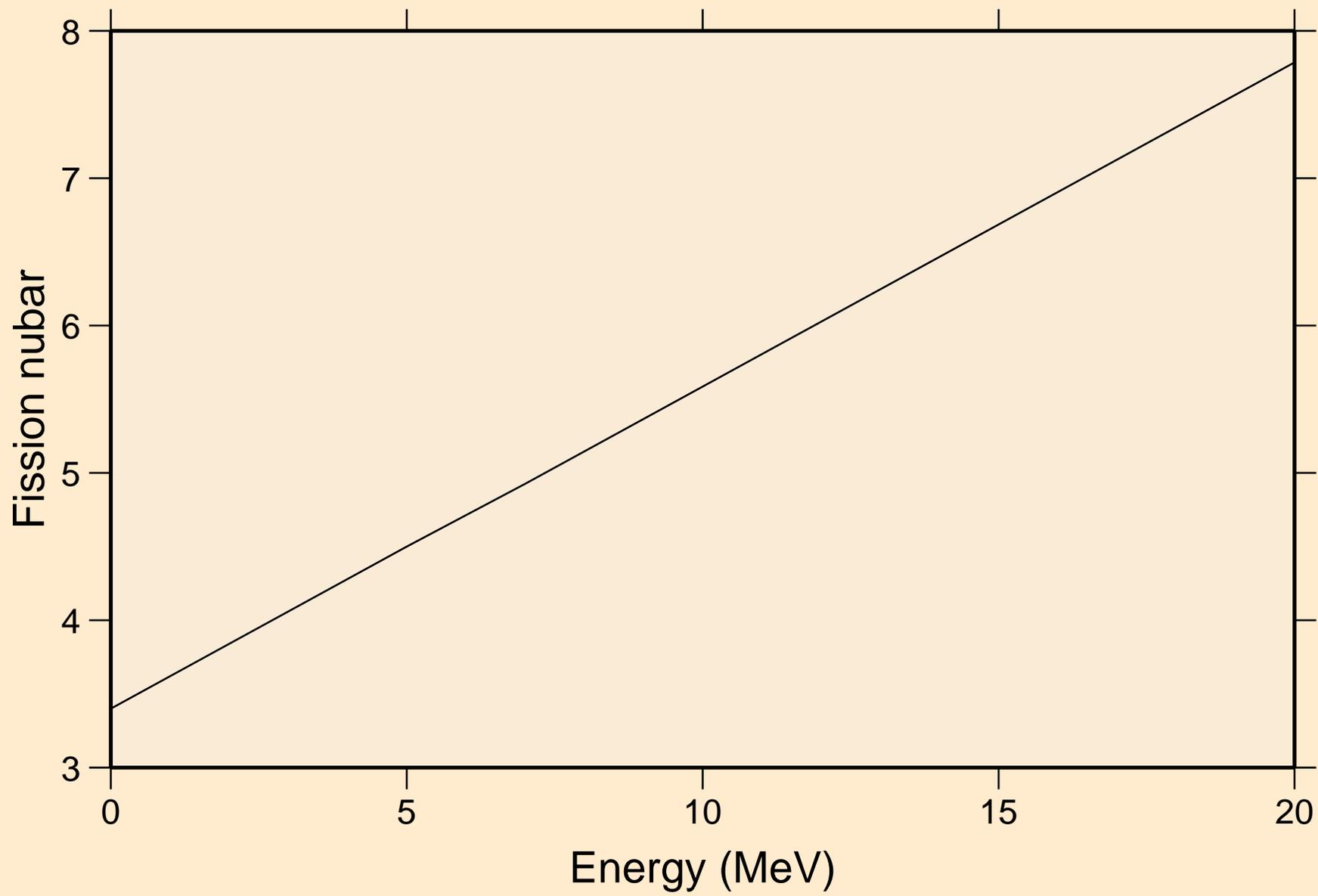


ENDF/B-VII CM-250  
angular distribution for (n,n\*c)

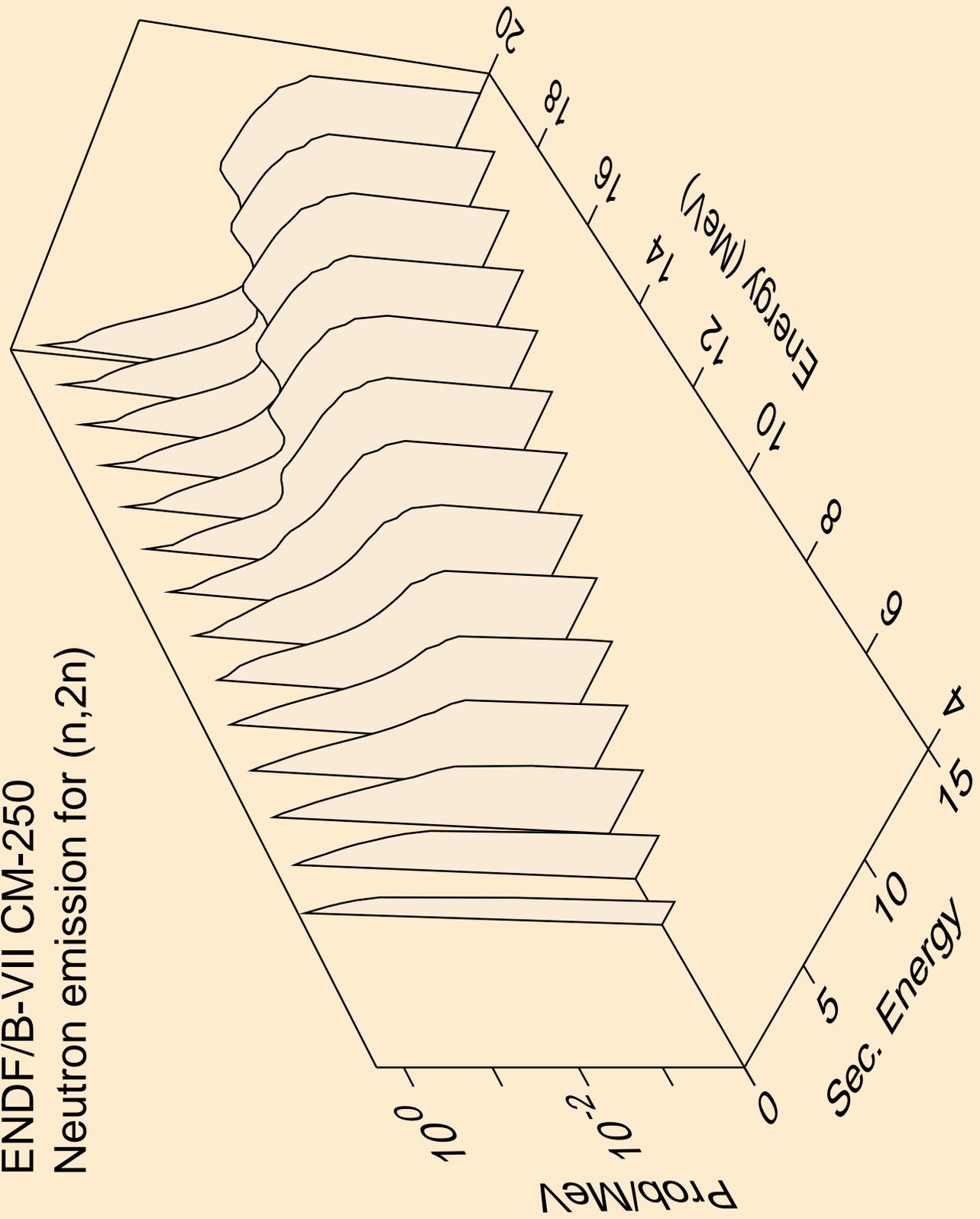


ENDF/B-VII CM-250

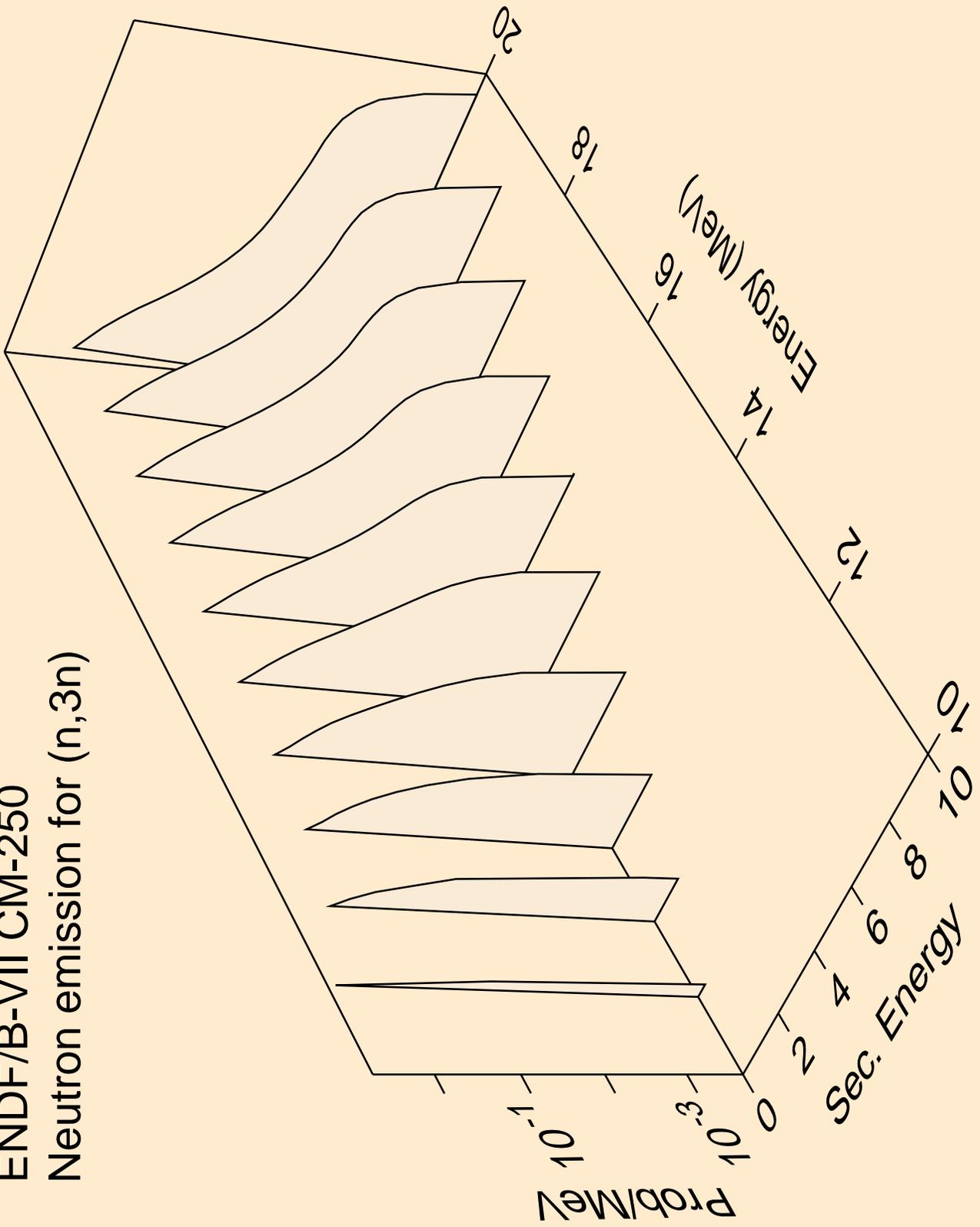
Total fission nubar



ENDF/B-VII CM-250  
Neutron emission for (n,2n)



ENDF/B-VII CM-250  
Neutron emission for (n,3n)



ENDF/B-VII CM-250  
Neutron emission for (n,n\*c)

